

秦山核电厂主泵轴卡死事故的堆芯DNBR计算

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摘要 用子通道分析程序THAS-PCI(COBRA-IV修改微机版,内含4种CHF关系式),对泵轴卡死事故做了堆芯最热燃料组件的DNBR计算。结果表明,W-3、BAW-2和CONDIE关系式计算值与CHF表值的 DNBR很接近,它们都在2s左右达到最小DNBR,并低于限定值1.3。BIASI式计算值比它们都高得多,远大于限定值1.3。

关键词 [子通道分析](#) [临界热流密度](#) [DNBR](#) [泵轴卡死事故](#)

分类号

DNBR CALCULATION IN CORE DURING COOLANT PUMP ROTOR SEIZURE ACCIDENT FOR QINSHAN NUCLEAR POWER PLANE

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Abstract DNBR of the hot fuel assembly in core is calculated by use of the subchannel analysis code THAS-PC1 for the coolant pump rotor seizure accident. The results show that DNBR calculated by W--3,BAW--2 and CONDIE correlations agree with the values of CHF table. They reach the minimum DNBR when the transient time is equal to 2 seconds, and the minimum DNBR fall below limiting value 1.3. The DNBR calculated by BIASI correlation is much higher and far greater than the limit value 1.3.

Key words [Subchannel analysis](#) [Critical heat flux](#) [DNBR](#) [Pump rotor seizure accident](#)

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