反应堆工程

MCNP程序用热中子散射数据制作和检验

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大, k_{eff} 平均偏差约65 pcm。

 关键词
 ENDF/B-\/II.0
 热中子散射
 MCNP
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Generating and Validation of Thermal Neutron Scattering Library for MCNP

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Abstract A continuous neutron transport library for MCNP and several temperature-depend ent multigroup cross section libraries with Bondarenko self-shielded factors based on ENDF/B-VII.0 were generated in former work. In the work the thermal $S(\alpha, \beta)$ tables for MCNP code were converted from the thermal neutron scattering sublibrary of ENDF/B-VII.0 using NJO Y program. The available thermal $S(\alpha, \beta)$ library named sab2002 (from ENDF/B-VI) release d with MCNP were investigated and compared with the self-produced library thb70, and some significant improvements of important medias, such as H in H₂O and D in D₂O, were presented. Forty-eight critical benchmark models were selected to calculate the $k_{\rm eff}$ using sab2002 and thb70, respectively. The results show that there is a not too much difference (65 pcm averagely) between the two libraries.

Key words ENDF/B-VII.0 thermal neutron scattering MCNP integral validation DOI

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